NUCLEAR-PHYSICAL CHARACTERISTICS OF MEPTUNIUM ISOTOPES.

S.S.Kovalenko, Yu.A.Selitsky, V.B. Funshtein, S.V.Khlebnikov, V.A. Yakovlev V.G.Khlopin Radium Institute, Leningrad 197022, USSR

Abstract: A survey is given of the work carried out at the Radium Institute on studying the nuclear-physical properties of neptunium that influence the work of nuclear power installations. There are presented the measured cross-sections of the reaction $^{237}\mathrm{Np(n,2n)}$, giving rise to the short-lived isomer $^{236}\mathrm{Np(s)}$, at E_{me} 7+15 MeV and on $^{252}\mathrm{Cf}$ spontaneous fission neutrons, the cross-sections of $^{237}\mathrm{Np}$, $^{236}\mathrm{Np(s)}$ and the long-lived isomer $^{236}\mathrm{Np(1)}$, by thermal neutrons and the respective resonance fission integrals, the cross-sections of $^{236}\mathrm{Np(1)}$ fission by neutrons in the energy range E_{n} = 0.0253 eV + $^{236}\mathrm{Np(1)}$ fission by neutrons in the energy range E_{n} = 0.0253 eV + $^{236}\mathrm{Np(1)}$ fission of reactions of alpha radiation. New data are given on the cross-sections of reactions $^{236}\mathrm{U(p,xn)Np}$ at E_{p} = $^{236}\mathrm{Np(1)}$ MeV applied for production of neptunium isotopes and plutonium-236.

(neptunium, isomer, neutrons, reactions, fission, cross-section)

Introduction

A series of works studying nuclear—physical properties of neptunium that influence the work of power installations have been carried out at the Radium Institute. A closed fuel cycle with broad reproduction of nuclear fuel is a characteristic feature in exploitation of fast—neutron nuclear reactors. The closed fuel cycle includes storage, transportation and reprocessing of spent fuel assemblies. In the nuclear reactor fuel there is 236Np arising as a result of the 237Np(n,2n) reaction in two isomeric states: the short-lived 236Np(s) with the half-life T1/2 = 22.5+0.4 h and spin I = 1 /1/, and the long-lived 236Np(1) with T1/2 = 1.55%10 years and I = 6 /2/. The decay chain of 236Np including 236Pu and 232U causes formation of radioactive gases and nuclides with hard gamma radi—ation, which hinders regeneration of plutonium and uranium fuel. To solve the problem of calculating the accumulation of 236Pu and 232U in nuclear fuel it was necessary to measure the cross-sections of the 237Np(n,2n) reaction and the cross-sections of 236Np and 237Np fission by thermal and resonance neutrons.

A practical interest to studying the 236Np(1) fission also arises due to this nuclide's having a great (~3000 b /3-7/) cross-section of fission by thermal neutrons, 6th, and being accumulated in sufficient amounts it could be used as an energy source.

236 Besides, the great value of 6th

an energy source.

Besides, the great value of of for 236Np(1) can cause systematic error in measuring oth (237Np) if neptunium has even a slight (10-5+ 10-6atom/atom) admixture of 236Np(1). Therefore it was necessary to check if the previously published values of oth (237Np) hadn't been distorted by the probable presence of 236Np(1) admixture in the used preparations.

From the scientific view-point the study of ²³⁶Np isomers enables to find out: 1) how nuclei with the same nucleo-

nic composition differing only by the spin manifest themselves in fission; 2) the cause of great values of the cross-sections of odd-even nuclei's fission by thermal neutrons; 3) the energy dependence of the isomeric ratio - the ratio of the cross-sections of ²³⁶Np(s) and ²³⁶Np(l) formation.

Production of ²³⁶Np.

The study of \$236 Np properties is possible only in the case if there is a sufficient amount of purified substance. 236 Np may be produced by many nuclear reactions, such as \$235 U(d,r),235 U(d,p2n),236 U(d,2n),238 U(p,3n),238 U(d,4n),237 Np(d,2n),237 Np(d,p2n),237 Np(d,t). The reactions with \$237 Np don't enable to reach the necessary purity of neptunium-236 because of impossibility to separate \$236 Np chemically from the material of the target. \$236 Np(1) was produced by the \$238 U(p,3n)\$ reaction having the greates of all the reactions with uranium yield (\$\sim 0.3\$ ng/\$\sup\$Ah\$ h at \$E_p = 30\$ MeV for a thick metallic uranium target). It was necessary to get \$236 Np(s)\$ with a small impurity of the short-lived neptunium isotope \$238 Np(T1/2= 2.1 days)\$ that has a great cross-section of Accumulation of \$236 Np(s)\$ with negligibly small (\$\leq 0.5 \%\$) impurity of \$238 Np\$ was done by the reaction \$236 Np(1)\$ nuclei was the number of \$236 Np(1)\$ nuclei was t

The number of ²³⁶Np(1) nuclei was determined by two methods: on the base of mass-spectrometric analysis of the isotopic ratio ²³⁶Np(1)/²³⁷Np with measuring the alpha activity of ²³⁷Np, and by measuring the gamma activity of ²³⁶Np(1) (Ey = 158 and 160 keV). The amount of the ²³⁶Np(s) formed was determined by the alpha radiation of the ²³⁶Pu daughter nucleus. For determination of the yield of ²³⁶Np(1) in chemical separation the tracer ²³⁸U(p,4n).

The number of ²³⁵Np nuclei was measured by its gamma and alpha activity. Because of considerable discrepancy of the values of partial times of ²³⁵Np alpha decay (about twice) in literature, its value was defined more exactly and turned out to be (4.2±0.2)x10⁴ years /8/.

In fig.1a the cross-sections of neptunium isotopes' formation during ²³⁶U irradiation with protons /9/ are shown. The solid and the dash curves represent the results of calculation within the

In fig.1a the cross-sections of neptunium isotopes' formation during 238U irradiation with protons /9/ are shown. The solid and the dash curves represent the results of calculation within the framework of the statistical model by the modified variant of the STAPIF program with account of the contribution of pre-equilibrium processes. A good agreement between the experimental data and the theoretical calculations attracts attention.

attention. Fig.1b presents new data on the isomeric ratio R = 6(s)/6(1) in the reaction $^{23}\text{SU}(p,3n)^{236}\text{Np}$. The decrease of R with the growth of the proton energy is qualitatively comprehensive, because with an increase of E_p the introduced angular momentum I grows and the formation of the high-spin state of $^{236}\text{Np}(1)$ becomes more probable.

Cross-sections of the reaction 237 Np(n,2n)

The cross-section values $\mathcal{O}_{n,2n}(s)$ of the reaction $^{237}\mathrm{Np}(n,2n)^{236}\mathrm{Np}(s)$ are important for calculating the accumulation of $^{236}\mathrm{Pu}$ and $^{232}\mathrm{U}$ in the reactor fuel. Measurement of these cross-sections is hindered by the small number of $^{236}\mathrm{Np}(s)$ nuclei formed. Earlier the

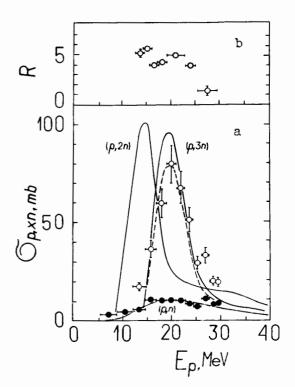


Fig.1: a - Experimental cross-sections of the reactions ²³⁸U(p,n)²³⁸Np (•) and ²³⁸U(p,3n)²³⁶Np(s) (•) and the calculated cross-sections of the reactions ²³⁸U(p,xn)²³⁹-*Np, x=1+3 (continuous curves) and ²³⁸U(p,3n)²³⁶Np(s) (dashed curve); b - Isomer ratio R = s/l of the reaction²³⁸U(p,3n)²³⁶Np.

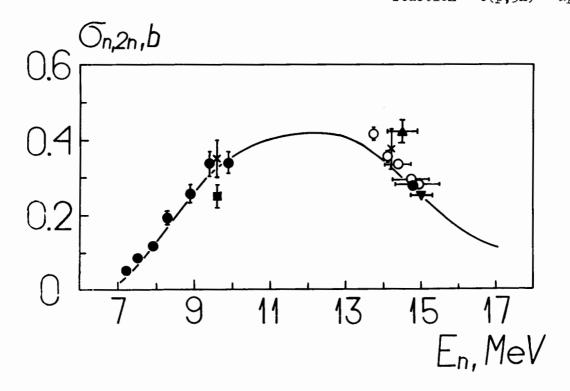


Fig.2. Experimental cross-sections of the reaction $^{237}\text{Np}(n,2n)^{236}\text{Np}(s)$ (\$\lambda - /10/, \$\lambda - /11/, \$\forall - /12/, \$\times - /13/, \$\vec{\pi} - /14/, \$\lambda - /15-18/\$) and the calculated dependence of this reaction's cross-section from work /19/ (continuous curve).

 $\sigma_{n,2n}(s)$ were determined only in the region $E_n = 14+15$ MeV /10-12/(excluding the only measurement at $E_n = 9.6$ MeV /13,14/), where in the experiment great neutron fluxes are attainable. First we measured the $G_{n,2n}(s)$ at $E_{n}=14.8$ MeV, using both the traditional method of determining the number of $^{236}\text{Np}(s)$ nuclei by the alpha radiation of the daughter 236Pu, and the developed new methods by means of registration of 236U gamma radiation in decay of 235Np(s) (Ey = 642 and 688 keV) /15,16/. Then a series of experiment was carried out for determination of $G_{n,2n}(s)$ in the range $E_n = 7+10$ MeV, where the yield of = 7+10 MeV, where the yield of 236Np(s) for the reactor neutron spectrum is maximum /17,18/. The neutron flux

rum is maximum /17,18/. The neutron flux was determined by means of the reactions 2/Al(n,a), 238U(n,f) and 238U(n,2n). The results of these and earlier /10-14/ experiments are presented in fig.2. The experimental values of 6n,2n(s) are well described by the results of the recently fulfilled calculations of these cross-sections /19/(the curve in fig.2). The value of the cross-section 6n,2n(s) was measured for 252Cf spontaneous fission neutrons /20/. The neutron flux was determined by means of activation detectors (2/Al, 197Au, 48Ti, 58Ni), that have threshold energies close in value to the threshold of the 237Np(n,2n) reaction. The value of 6n,2n(s) for 252Cf spontaneous fission neutrons appeared to be 4.7±0.5 mb. A calculation of the cross-section averaged over the spectrum of 252Cf fission neutrons was done as a check of agreement of the resoured cross-sections on neutrons was done as a check of agreement of the measured cross-sections on monoenergetic neutrons and the integral cross-section. A numerical integration indicated a value of 3.5±0.6 mb which agrees with the integral experiment.

Cross-sections of neptunium nuclei neutron fission

The cross-sections of $^{236}Np(1)$, $^{236}Np(s)$ and ^{237}Np fission by thermal neutrons (with a Maxwell spectrum) , 6; and their resonance integrals of fission, If, were measured on the WWR-M reactor of the Leningrad Institute of Nuclear Physics. The measurements were done by the relative method using 239 Pu. During measurement of the resonance integrals the thermal neutrons were absorbed by a cadmium screen 1 mm were absorbed by a cadmium screen 1 mm thick. The fission rate of the short--lived isomer 236Np(s) decreased with time in accord with its known half-life T_{1/2} = 22.5 h. In table 1 there are the results of

those experiments. The given in table 1 value of 6^{th} for 237Np was calculated taking account of the Westcott multiplier for 237Np /25/ and corresponds to the neutron energy 0.0253 eV.

The experiments for determination of 6^{th} (237Np) and I_f (237Np) were carried

Table 1. Cross-sections of thermal neu-tron fission and the resonance integrals of neptunium nuclei

Nuclide	$oldsymbol{\sigma_{ ext{f}}^{ ext{th}}}$, b		Litera- ture
236 _{Np} (1)	2760 <u>+</u> 170	1030 <u>+</u> 100	/21,22/
236 _{Np} (2)	2740 <u>+</u> 140	690 <u>+</u> 360	/22,23/
237 _{Np}	0.020 <u>+</u> 0.001	4 .7 0 <u>+</u> 0.23	/24/

out twice: before and after a tenfold enrichment of neptunium with respect to the mass A = 237 on an electromagnetic mass-separator. The results of those experiments proved to be equal within the limits of the errors (±5%). The coincidence of the results of measuring 6th(237Np) with one another and with the previously known values /26,27/shows that these values are not distorted by a possible presence of the 236Np(1) admixture in the applied preparations.

The 236Np(1) fission cross-section

in the neutron energy range from 0.0253 eV to 0.3 cV was measured with a monochromator with a graphite crystal on the WWR-M reactor of the Leningrad Institute of Nuclear Physics, and in the range $E_n = 0.1$ eV + 20 keV, by means of a lead slowing down spectrometer on the linac "Fakel" at the I.V.Kurchatov In-

stitute of Atomic Energy (fig.3 /28/).
As one can see from fig.3, the energy dependence of the 230Np(1) neubron fission cross-section differs insig-

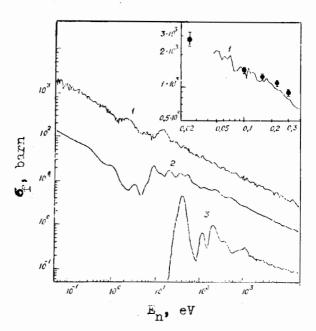


Fig. 3. Fission cross-section of ²³⁶Np(1) (curve 1). Curves 2 and 3 are ²³⁵U and ²³⁷Np fission cross-sections multiplied by coefficient which accounts their concentrations in the target.

nificantly from the 1/v-dependence in the whole energy range $E_n = 0.0253 \text{ eV} + 20 \text{ keV}$. The resonance integral of ± 20 KeV. The resonance integral to 20 KeV. The base of these data, proved to be 1040+60 b, which well agrees with its experimental value (table 1). The \mathfrak{S}_1 value for \mathfrak{S}_2 value for \mathfrak{S}_3 value for \mathfrak{S}_4 value \mathfrak{S}_4

tron energy spectrum (T=330 K).

As noted above, the thermal neutron fission cross-sections of odd-odd nuclei are greater than those of even-odd nuclei and Thermal the transfer of the section of the se clei. In fig.4 the known from literature values are presented of odd-odd and even-odd target nuclei thermal neutron fission cross-sections, including the data for 236Np(1) and 236Np(s) obtained by us. As seen from fig.4, the nuclides

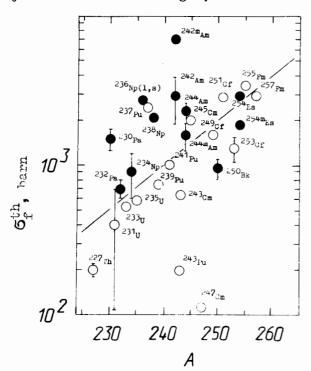


Fig. 4. Cross-sections of the thermal neutron fission of even-odd () and odd-odd (●) nuclei.

ere divided into two groups by a straight line, above which are the odd-odd nuclei fission cross-sections (above twice, on the average). The increased density of the compound nucleus's levels is one of the factors that may cause great cross--sections of the odd-odd nuclei fission. In work /29/ there was found no correlation between the Gth and the density of the levels of the compound nucleus in analysis of the whole collection of the odd-odd nuclei. Such an analysis becomes more correct if to consider pairs of isomers of the odd-odd nuclei having the same nucleon composition. The cross-section of the isomers of 242Am, 244Am, 254Es, 254Es, 254Es, 254Es, 254Es, 254Es, 255Es, 255 correlate with the spin dependence of the level's density, but the 6th for 236Np(s) and 236Np(l), measured by us, don't prove it. The relationship of 6th for 236Np(s) and 236Np(l) seems to be

determined mainly by such a casual element as the degree of closeness of the compound nucleus's resonance to the thermal energy of neutrons. Another factor, that can also cause great values of oth and I, is an increased dersity of the transition states on the fission ba-rrier and, hence, a great number of open fission channels of the formed odd-even compound nucleus. On the base of analysis of the 236Np(1) fission cross-section in the resonance region of the neutron energy, averaged values were obtained of the total (0.5+0.2 eV), the fissioning (0.45+0.16 eV) resonance widths and the number of the open fission channels N_r=21+8 (by an order of magnitude greater than the N_r for even-odd target nuclei).

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